

“A DETECTION SYSTEM AND A DETECTION METHOD BASED ON PULSED ENERGETIC PARTICLES”

Field of the invention

5 The invention relates to active neutron interrogation technology and more particularly to a detection system that uses neutrons and gamma photons to interrogate and detect materials or compounds in items.

Background of the invention

 So-called ‘neutronic interrogation’ is the only non-invasive technology
10 known to date which allow to extract, from a shielded and sealed item, a signature that is related to the chemical elements or compounds contained in the item.

 Such technology therefore brings significant progress over X-ray machines and the like which allow to discriminate between items only by
15 their shape and density of material, and is used for various applications, e.g. the detection of explosives, nuclear materials or contrabands such as narcotics in items or buildings.

 A number of neutron-based detection techniques have been developed, depending on the practical application.

20 Thermal Neutron Analysis (TNA) has been tried, for example, for inspection of checked baggage at airports. More specifically, low-energy neutrons cause nitrogen contained in certain explosives to emit gamma rays and cause fissile materials to give off neutrons of their own. However, the first-generation TNA screeners produced an unacceptable rate of false
25 alarms, due to a large number of nitrogen-bearing articles contained in a typical baggage. Furthermore, TNA requires moderators to slow down fast neutrons from a source to thermal neutrons.

 Pulsed Fast Thermal Neutron Analysis (PFTNA) of baggage for contraband such as explosives and narcotics has also been proposed. It
30 combines the detection of gamma ray emissions from several different neutron interactions in a single system and used a short pulse high-energy neutron to perform FNA (Fast Neutron Analysis) interrogation. This makes

possible to separate in time the FNA and TNA interrogations and improves the quality and statistics of the gamma signatures measured.

Typically, a single repetitively pulsed neutron generator based on the deuterium-tritium reaction is used to produce a short pulse (several μs) of 14 MeV neutrons. During this pulse, the interaction is primarily due to the inelastic scattering of high energy neutrons in the object being interrogated. The gamma ray emission is primarily composed of prompt gamma photons from $(n, n'\gamma)$ and $(n, p\gamma)$ reactions. The pulse is repeated with a frequency of 10 kHz and the spectra of the prompt gamma photons from the high energy neutron interaction are collected using conventional single photon counting gamma spectroscopy technique. From these spectra, gamma signatures from elements such as C and O can be extracted. Documents WO-99/53344A and "Vourvopoulos G. et al. "A Pulsed Fast-Thermal Neutron System for the Detection of Hidden Explosives", Nuclear Instruments & Methods in Physics Research, Section B: Beam Interactions with Materials and Atoms, Elsevier, Amsterdam, NL, vol. B79, n° 1/4, 2 June 1993, pp. 585-588, XP000381502 disclose a detection system of this kind.

In such system, between the neutron pulses, part of the fast neutrons continue to collide with the background material, in particular the light elements, and slow down to thermal energy.

When a neutron has an energy below 1 eV, it can easily be captured by elements such as H, N or Fe, producing prompt gamma photons from such (n, γ) capture reactions. A separate thermal neutron reaction spectrum is constructed from the gamma emission detected during this period. Document DE 10323093 A1 discloses such approach.

After a predetermined number of pulses, there is a long pause which allows the detection of gamma photons emitted from elements such as O, Al and Si which has been activated and decay with a delayed gamma reaction.

By cycling through these three steps, three separate spectra are generated, representing the fast neutron interaction, the thermal neutron interaction and the delayed activation interaction. These three spectra would

allow, in principle, a clear determination of all the four basic elements in high explosives.

In practice, the identification of a substance in PFTNA is performed by examining the atomic ratios, e.g. the ratio of carbon atoms to oxygen
5 atoms (C/O), as disclosed in document US-5200626A. This is done by taking the ratio of the intensities for carbon and oxygen gamma rays and then applying the ratio of the cross-sections for the gamma inducing reactions of these elements.

Unlike the FNA process with inherently mixed gamma signals from
10 the TNA process, the PFTNA process specifically separates the spectra due to the FNA and the TNA processes, which improves the identification of the elements of interest in explosive detection.

The short and specific time during which the gamma rays from FNA
process are emitted and collected improves the signal to background noise
15 ratio of the gamma signatures from C and O compared with conventional FNA technique.

This improvement, however, is significant only if there are very few thermal neutron capture reactions during the time of FNA process. However, in the context of concealed explosives, there is often a large
20 region, besides the small region occupied by the explosives, where a signal from thermal neutron capture reactions will be emitted. This will significantly deteriorate the signal to noise ratio, which in turn means that a long interrogation time is again necessary to obtain useful statistics.

Furthermore, in the context of luggage inspection, the presence of
25 large quantity of plastic materials from the outer casing to the content, will strongly affect the identification of explosives made from a measurement of the C/O or N/O ratio.

Another known interrogation technique is PFNA (Pulsed Fast Neutron Analysis) and is based on the velocity/energy relationship of a fast neutron,
30 whereby a very short and high energy neutron pulse of a few nanoseconds duration can provide an opportunity to record the spatial region of the

neutron interaction, from the time of flight (TOF) of the neutron through the object under interrogation.

In order to collect the TOF information, a mono-energetic fast neutron pulse is used, so that the spatial position of the neutron at any given time
5 can be calculated. When a fast neutron makes a gamma emitting collision with an element in the object under interrogation, it is then possible to associate the time of detection (and therefore emission) of this gamma photon with a given position in the object. By using a repetitively pulsed neutron source of such very short duration, and by measuring the time of
10 occurrence relative to the launch time of the neutron pulse, as well as the energy of the gamma emission, the elemental density of a range of elements inside a bulk material, along the path of the fast neutron, can be determined.

Specifically, using the prompt gamma reactions from fast neutrons,
15 the relative concentration of the elements C, N, O in a given spatial cube, called a voxel, can be determined. Using the specific ratios of these three elements in high explosives, the presence and the location of an explosive can be identified.

More particularly, by using suitable gamma detector arrays and by
20 scanning the neutron beam pulse along two axes, a 3D map of the element concentration in the different voxels can in principle be constructed.

The PFNA technique is particularly suitable for examining a small amount of explosives hidden within a large object of relatively low average density.

25 However, a very short mono-energetic neutron pulse is required if a reasonable spatial resolution is to be achieved. Typically, such neutron source is produced from a linear accelerator. Although such accelerator based PFNA systems have been developed for the examination of cargo containers and for air security applications with good results, the large size
30 and substantial shielding requirement associated with the linear accelerator means that the system cannot be mobile, so that the technique is not appropriate for demining or airport applications.

In addition, all the existing neutron-based detection techniques, while being accurate in terms of analysing the chemical contents of a sample, are slow (it takes typically several minutes to perform a single detection).

This bottleneck is the result of the standard photon counting technique required to create the gamma spectrum build-up and hence to
5 allow the chemical elements in the sample to be detected.

The reason is that the detector can receive only one photon at a time and must wait for a detection time window to analyse the energy of the photon before the next photon can arrive.

10 With a very high intensity neutron pulse, it is not possible to record each activation gamma photon or backscattered neutron individually, as the phenomenon of pulse piling-up will occur. Indeed, if more than one neutron is produced in a given measurement time window, the photons are considered as arriving "together" and therefore are not discriminated from
15 each other.

The maximum rate of single photon detection and analysis, therefore limits the maximum neutron fluency usable in a single pulse.

This leads, as cited above, to a detection time of several minutes to construct useful spectra with a sufficiently high signal to noise ratio for
20 positive detection. The reason is that 10^6 or more measurements, each typically requiring 1 microsecond or more, are needed to build up a good quality spectrum.

Moreover, the neutron source used in such known systems involves radioactive elements or electrical generators using radioactive targets,
25 which is highly undesirable in particular in civil environments.

Summary of the invention

The present invention seeks to overcome these shortcomings of conventional neutron-base detection techniques for inspecting items.

30 More particularly, in addition to the self-evident considerations of cost-effectiveness and acceptable footprint, a system offering trustworthiness and very fast detection time would be highly desirable.

It is an object of the present invention to provide a new material detection technology based on impulse neutron interrogation methodology which avoids the conventional single photon counting method to characterize the photons and neutrons, activated and backscattered from an
5 object.

It is a further object of the present invention to provide a versatile detection system covering a wide range of energies, including their evolution in time, with a single measurement.

Another object of the present invention is to provide an identification
10 system allowing to significantly reduce false alarm rates and to provide improved data output for operator decision.

It is also an object of the present invention to provide a system capable of detecting both conventional contrabands like narcotics and certain nuclear materials, which may be hidden in metallic containers that
15 are opaque to conventional X-ray inspection systems.

It is still a further object of the present invention to provide a detection system which overcomes the limitation due to pulse piling-up.

Still another object of the present invention is to provide a detection system allowing to detect the presence of bulk explosives in airport baggage
20 or of landmines in demining by means of a single high intensity neutron pulse and with a short response time.

Accordingly, the present invention provides a detection system, comprising :

- a particle source for generating a pulsed flux of energetic particles
25 including both neutrons and gamma photons and for directing said flux towards an item to be analyzed, said particles being intended to react with nuclei of material(s) in said item,

- a detection unit comprising at least three detector assemblies responsive to neutrons and gamma photons in respective energy ranges
30 coming from said item and impinging thereon in response to said flux of energetic particles, wherein the detector assemblies are arranged to

operate in current detection mode to deliver current signals representative of impinging gamma photons and neutrons over time, and

- a data processing unit connected to the outputs of said detectors, capable of generating a signature from said signals following the application of said pulsed flux to said item, including time-related signal features, and for comparing said signature with stored reference signatures.

Preferred but non-limiting aspects of this system are as follows:

- * each detector assembly comprises a respective energy band-pass filter.

- * each detector assembly comprises a scintillator coupled to a photomultiplier by means of a set of flexible fiber optics cables.

- * said neutrons and gamma photons source comprises:

- first and second electrodes,
- a plasma ion source at the first electrode,
- a plasma ion source driver for allowing a ion plasma containing deuterons to develop towards the second electrode,
- means for applying between said electrodes a short high voltage pulse at a time at which said ion plasma is in a transitional state with a space distribution of deuterons at a distance from said second electrode, so as to accelerate said deuterons towards said second electrode while overcoming the space charge current limit of a conventional vacuum diode, and
- said second electrode forming a lithium-bearing target, so as to generate said neutrons at said second electrode by deuteron/lithium interaction, wherein neutrons interact with said target to produce gamma photons.

- * said neutrons have an energy of at least 3 MeV, which is appropriate for the detection of nuclear materials.

- * said neutrons have an energy of at least 5 MeV, which is appropriate for the detection of carbon-based materials, while not interacting with possible surrounding elements.

* said neutrons have an energy of at least 8 MeV, which is appropriate for the detection of explosives, as they interact with all four elements H, C, N, O common to most explosives.

* the signals delivered by the detector assemblies correspond to one
5 single pulse of particles combining neutrons and gamma photons from said source.

According to a second aspect, the present invention provides a method for detecting properties of materials, substances or compounds
10 contained in items, comprising the following steps:

- applying a pulsed flux of energetic particles including both neutrons and gamma photons to an item to be analyzed, said particles being intended to react with nuclei of material(s) in said item,
- detecting neutrons and gamma photons coming from said item in
15 response to said application in at least two different energy ranges and in current detection mode so as to deliver current signals representative of impinging gamma photons and neutrons over time,
- delivering time-resolved current signals representative of neutrons and gamma photons thus detected,
- 20 - generating from time-related features of said signals a signature, and
- comparing said signature with stored reference signatures.

Brief description of the drawings

25 The present invention will be better understood from the following exemplary description of a preferred embodiment thereof, made with reference to the appended drawings in which:

- Figure 1 is a block diagram of an identification system according to the present invention;
- 30 - Figure 2 illustrates in greater detail a detection unit comprised in the system of Figure 1; and

- Figure 3 is a time chart illustrating a typical signal collection in relation with pulse emission and their arrival at the item to be analyzed.

Detailed description of a preferred embodiment of the invention

- 5 Now referring to the drawings, an identification system 700 according to the present invention is depicted in the figure 1 and comprises:
- a conveyor system 650 for items 600 to be inspected;
 - an energetic particle source 500;
 - a detection unit 400; and
 - 10 - a data processing unit 800.

The source 500 generates a high flux and a high intensity of short pulse energetic particles including both neutrons and gamma photons directed toward an item 600 to be checked.

- 15 The source includes a particle generator 100 which generates the flux of energetic particles in response to a pulsed power unit 200 controlled by a control unit 300.

In operation, a short pulse of high density, high flux energetic particles is generated and directed towards an item 600 placed in an inspection region towards which a beam collimator 130 is directed.

- 20 The conveyor system 650 can be conventional and is used for moving the items 600 such as pieces of baggage by appropriate increments through the inspection region.

- 25 The source 500 is used in combination with the detection unit 400 to detect gamma photon and neutron signals representative of the materials contained in the inspected item 600, upon interaction with the penetrating source of energetic particles.

- 30 As shown in Figure 2, the detection unit 400 comprises an array of detectors 410 having a wide set of energy responses and capable of detecting both gamma photons and neutrons backscattered from item 600 when exposed to the source 500. Each detector 410 is sensitive to a given range of energy for the gamma photons and neutrons backscattered from the object.

The detectors together provide data which can be processed at unit 800 to give a unique signature for specific chemical or nuclear materials or compounds to be looked for.

Correlatively, a detection according to the present invention is based
5 on characteristic gamma signature recognition which can be acquired during a single pulse of energetic particles (although multiple pulse detection is also possible) and does not require detailed energy-resolved spectrometry contrary to prior art systems as disclosed in WO-99/53344-A and DE-103 23 093-A1.

10 The data processing unit 800 analyzes the signals provided by the detector array in order to generate the gamma signature of item 600. Then a statistical comparison of the calculated signature with a database of reference signatures is used to make the decision on the presence or absence of certain materials or compounds in said item.

15 A more detailed description of the various components of the system will now be given.

Particle source 500

The particle generator 100 is driven by a pulsed power supply unit 200 to generate short pulses 140 of energetic particles.

20 These pulses 140 are generated on demand upon a control trigger delivered by a control unit 300. At all other times, the whole system 500 is in an "off" condition.

The particle generator 100 is contained in a vacuum chamber 150 containing a pair of spaced electrodes, i.e. an emitting electrode 110 and a
25 target electrode 120.

Typically, the distance between the two electrodes 110 and 120 is a few centimetres and the pressure is between 0.1 and 10 Pa.

A high voltage driver 220 for the emitting electrode 110 is provided in the power supply unit 200 and is used to power said electrode by applying a
30 suitable voltage pulse 225 between a pair of electrode members (not shown) belonging to said electrode and forming a plasma discharge ion source.

A low pressure plasma with a plasma density having an order of magnitude of 10^{13} particles/cm³ or more is thus created in the vicinity of electrode 110 and then develops a space distribution of charged particles within chamber 150. After a predetermined time delay dt , a high voltage pulse 215 generated at pulse generator 210 also provided in power supply unit 200 is applied between electrodes 110 and 120 in order to accelerate towards the second electrode 120 the particles having a predetermined charge sign contained in the plasma.

The time delay dt is selected as a function of the voltage level of the plasma triggering pulse 225, the accelerating voltage pulse 215, the geometry of the diode formed by the two electrodes 110 and 120 and the pressure within the chamber 150.

The control unit 300 is capable of triggering driver 220 and then generator 210 according to the above time delay.

The synchronised command control the high voltage pulse supply 210 to start applying a suitable pulse voltage 215, after a time delay dt , between the two electrodes 110 and 120 so that a charged particles beam is extracted from the plasma .

In an embodiment of the present invention, the high voltage pulse generator 210 comprises in a manner known per se a voltage multiplication circuit followed by a pulse compression circuit (not shown).

More particularly, a mains voltage source such as 220 V, 50 Hz is first increased to 30 kV using a conventional electronic inverter unit. This voltage is used to feed a 4 stage Marx circuit responsive to a trigger control to produce a voltage pulse of 120 kV. This voltage is then used to charge a pulse shaping circuit to produce a 5 ns pulse of 120 kV. The output of this pulse shaping circuit is coupled to a pulse transformer, providing a final 5 ns voltage pulse 215 of 720 kV.

Upon application of pulse 215, charged particles contained in the plasma are accelerated to form a charged beam with a high current (typically more than 1 kA) which impinges on the electrode 120 serving as target electrode with an energy which can reach 500 keV or more, thereby

producing a flux of highly energetic particles as the result of a charged particle-induced nuclear reaction.

It will be appreciated here that the principle of operation of source 500, where a high-energy flux of charged particles is produced by the direct application of a ultra-short high voltage pulse 215 to electrodes between which an ion plasma is in a transitional state, allows to overcome the space charge current limit of a conventional vacuum diode. For instance, a short pulse (<10 ns), high current (> kA), high-energy (> 700 keV) charged particle beam can be generated.

With the particle source as described above, the flux 140 of energetic particles 140 is emitted in an isotropic manner. In order to generate a beam directed towards the item 600 to be analyzed, a suitable collimator 130 is provided.

Finally, it will be noted that the control unit 300 may also serve as a monitoring unit, providing control and status information on all modules of source 500. For this purpose, unit 300 is coupled to a set of safety sensors and/or detectors to ensure safety interlock and proper operation of source 500.

For high speed screening operations, the source 500 can be repeatedly activated, e.g. one to several times per second.

The source 500 may be used for generating various types of energetic particle beams. In a preferred embodiment of the invention, such particles are neutrons and gamma photons, which are generated by the impact of an energetic charged deuteron particle beam of around 10 ns duration at current value having an order of magnitude of kA, on a lithium alloy target electrode 120, thereby producing a 10 ns pulse 140 of more than 10^8 neutrons, thus providing a high equivalent fluence rate of 10^{16} neutrons/second with a broad energy distribution up to 14 MeV.

The source 500 as described in the foregoing allows to detect substantially all gamma photons and neutrons backscattered by the item over a very short duration.

By comparison, a conventional sealed neutron tube as described e.g. in document US-5 200 626-A typically provides a small quantity of neutrons in 10 microsecond bursts which have to be repeated at a frequency of 1 kHz or more to obtain an equivalent fluency rate of 10^8 n/s, and several minutes
5 of operation are required to provide sufficient gamma photon data for analysis. This leads to a radiation dose rate at least 2 orders of magnitudes higher than the one occurring with the present invention.

In a non limiting example, at a distance of 3 m, an item 600 with a 10 cm² exposed area will receive a neutron flux at an equivalent rate of 10^{11} n/s
10 from the single pulse obtained with the source 500 of the present invention.

Detection unit 400

As shown in Figure 2, the detection unit 400 comprises a gamma photon detection part 410 comprising an array of detectors (three in the
15 present example) 411, 412, 413.

Each detector is connected to a photomultiplier 431, 432, 433 by means of a respective set of flexible fiber optics cables 421, 422, 423.

Each detector preferably comprises a conventional plastic scintillator (e.g. of well known NE102A type) of appropriate surface area selected so as
20 to be sensitive both to gamma photons and neutrons, and outputs a signal representing the gamma photons and neutrons backscattered from the item 600 when exposed to the beam 140 generated by source 500.

The size of each scintillator is for instance 180 mm x 180 mm x 25 mm. More generally, a large size of the scintillators allows to substantially
25 improve signal/noise ratio.

Each detector 411, 412 and 413 has a response within a specific energy spectrum, and this is preferably obtained by placing materials (not shown) on the travel path of neutrons and gamma photons from the item 600 to the respective detector, these materials acting as energy band-pass
30 filters in different energy ranges for each detector 411, 412 and 413.

The output amplitude of each detector is therefore related to the spectral content within that range, and detectors 411, 412 and 413 provide

respective signals A, B and C indicative of a quantity of received radiation/particles.

It will be appreciated that, by connecting the scintillators to photomultipliers via flexible fibre optics cables of appropriate length, the
5 photomultipliers can be placed at a distance from the source of radiation and be shielded against the effect of energetic particles and electromagnetic radiation generating noise in said photomultipliers.

Preferably, the photomultipliers 431, 432 and 433 operate in current detection mode to allow real time measurement of the evolution of the
10 photons/particles impinging on the respective scintillators.

In this regard, instead of measuring the energy of individual gamma photons or neutrons, the current signal produced by the collection of particles and photons arriving at a given detector 411, 412 or 413 is recorded as a function of time.

15 Therefore, the conventional issue of pulse pile-up when particles impinge on the scintillator within a very short time interval, resulting in counting errors, is avoided.

The signal outputs from the photomultipliers 431, 432 and 433 are supplied to analog-to-digital converters globally referenced 440 so as to
20 provide a corresponding digital data stream representative of the photons/particles received from irradiated item 600 as a function of time in different energy bands.

As an example, a 4-channel Tektronix TDS3034 transient digitizer with a maximum sampling rate of 2.5GS/s and a maximum data memory
25 depth of 10 k samples can be used.

Preferably, the digitalized data are recorded for e.g. 20 μ s after each particle pulse 140 is applied to the item, with a sampling rate of 2 ns at most so as to keep within the memory depth of the digitizer.

The signals provided by the detection unit 400 combine the results
30 from any neutron interaction modes in the materials of item 600, including elastic, inelastic and captured reactions, that occur in response to the single

ultra-short high energy neutron pulse 140 and that lead to the emission of gamma photons or neutrons.

More particularly, the detection unit 400 detects prompt and delayed gamma photons generated from fast and thermal neutrons, as well as
5 neutrons backscattered or emitted from the sample, during different periods after the single short neutron pulse is triggered.

An example of a typical timing of signal detection in relation with neutron pulse generation is given in figure 3.

In this example, a single pulse P1 of about 10^8 neutrons having a
10 duration of 10 ns is emitted in a 4π solid angle. A secondary pulse of gamma photons P2 is also emitted with a slight time shift and a slightly longer duration. This gamma pulse P2 is generated as a result of the interaction of the neutron pulse P1 with the immediate surrounding of the neutron-generating target, as well as within the target itself.

15 The blocks A1 and A2 correspond respectively to the neutrons of pulse P1 arriving on a target located 1 metre away from the source, and to the gamma photons of pulse P2 arriving on said target.

The detected signals as shown in the bottom of Figure 3 are as follows:

- 20 - initially, the collected signals S1 are due to a fraction of the gamma pulse P2 arriving directly at the detectors;
- this is closely followed by the signals S2 which originate from a certain quantity of the gamma pulse P2 being backscattered from item 600;
 - some few tens of nanoseconds later, the received signals S3 are
- 25 composed of high-energy gamma photons produced when high energy neutrons travel directly through item 600 and have non-elastic collisions with the nuclei of the item material(s); the very short neutron pulse P and the relatively slow speed of travel (a 10 MeV neutron travels about 4.4 cm in one nanosecond) allow good spatial discrimination; in addition, the high
- 30 intensity of the neutron flux leads to a good signal to noise ratio from the detectors;

- the latter part of signals S3 are gamma photons generated from the interaction with item 600 of a large quantity of delayed neutrons resulting from the scattering of the uncollimated neutrons at the source;

- these signals are followed (time scale of the order of the
5 microsecond) by signals S4 corresponding to neutrons backscattered from the item;

- the last detected signals (after ten microseconds or more, not shown) result from captured gamma photons produced from neutrons which have been "thermalized" in the source collimator, as well as from the sample
10 and its surrounding. These thermal neutrons are captured by the nuclei of the item material(s) which in turn generate gamma photons.

Many of these interactions are applicable for a wide variety of elements/compounds. Therefore, by recording the detected signals over an long period of time (typically tens of microseconds) after the start of the
15 neutron pulse 140, very meaningful signals are obtained, esp. when considering evolution over time.

Preferably, a gain adjustment circuit is incorporated into each of the detector assemblies (scintillator + fibre optic + photomultiplier) in order to compensate for the variations in the coupling efficiency between the
20 scintillator and a fibre optic.

Moreover, the detector assemblies are advantageously cross-calibrated by taking a set of measurements without the energy filters in front of the detectors and then with identical energy filters on all detectors.

All detection units are calibrated in the same way and the gain of
25 each unit is adjusted so that the variation in signal output between all units is within a given range (e.g. a factor of two at most).

The calibration can be made with a reference pulsed light source coupled to a plurality of fibre optics each coupled to a respective photomultiplier. In this way, each photomultiplier will be illuminated by the
30 same calibration light source through identical fibre optics coupling.

An automatic calibration process can also be conducted with one or several samples of well defined materials, for example organic materials

such as Melamine and Polythene. Such process allows to compensate for possible drifts in the detectors sensitivity.

Data Processing unit 800

The data processing unit 800 comprises suitable signal processing
5 power for the plurality of signals A, B and C received from detection unit 400.

Preferably, such processing involves applying a predetermined set of algorithms to said signals, including their evolution over time, in order to generate a signature associated with each item 600 analyzed.

10 Once an item has been analyzed, its signature is applied to a database for comparison with a set of reference signatures corresponding to different known materials or compounds or substances, so that the presence of any such materials or compounds or substances can be quickly identified and possibly quantified.

15 Preferably, unit 800 is programmed so as to provide an operator with simple yes/no answers for different types of materials or compounds or substances in a very short response time (typical processing times will be from a fraction of second to several seconds with state of the art processing capability).

20 While the present invention has been described with respect to exemplary embodiments thereof, it will be understood by the skilled person that many variations and modifications can be made thereto.

As indicated in the foregoing, although typical applications of the present invention are security checks for airport baggage and detection of
25 buried objects such as landmines and antipersonnel mines, the present invention may have numerous other applications.

CLAIMS

1. A detection system, comprising :
 - 5 - a particle source (500) for generating a pulsed flux (140) of energetic particles including both neutrons and gamma photons and for directing said flux towards an item (600) to be analyzed, said particles being intended to react with nuclei of material(s) in said item,
 - a detection unit (400) comprising at least three detector assemblies
10 (411, 421, 431; 412, 422, 432; 413, 423, 433) responsive to neutrons and gamma photons in respective energy ranges coming from said item and impinging thereon in response to said flux of energetic particles, wherein the detector assemblies are arranged to operate in current detection mode to deliver current signals representative of impinging gamma photons and
15 neutrons over time, and
 - a data processing unit (800) connected to the outputs of said detectors, capable of generating a signature from said signals following the application of said pulsed flux to said item, including time-related signal features, and for comparing said signature with stored reference signatures.
20
2. The system of claim 1 , wherein each detector assembly comprises a respective energy band-pass filter.
3. The system of any one of claims 1-2, wherein each detector
25 assembly comprises a scintillator (411; 412; 413) coupled to a photomultiplier (431; 432; 433) by means of a set of flexible fiber optics cables (421; 422; 423).
4. The system of any one of claims 1-3, wherein said neutrons and
30 gamma photons source comprises:
 - first and second electrodes (110, 120),
 - a plasma ion source at the first electrode,

- a plasma ion source driver (220) for allowing a ion plasma containing deuterons to develop towards the second electrode,
- means (210) for applying between said electrodes a short high voltage pulse at a time at which said ion plasma is in a transitional state with
5 a space distribution of deuterons at a distance from said second electrode, so as to accelerate said deuterons towards said second electrode while overcoming the space charge current limit of a conventional vacuum diode,
- said second electrode (120) forming a lithium-bearing target, so as to generate said neutrons at said second electrode by deuteron/lithium
10 interaction, wherein neutrons interact with said target to produce gamma photons.

5. The system of any one of claims 1-4, wherein said neutrons have an energy of at least 3 MeV.

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6. The system of any one of claims 1-4, wherein said neutrons have an energy of at least 5 MeV.

7. The system of any one of claims 1-4, wherein said neutrons have an
20 energy of at least 8 MeV.

8. The system of any one of claims 1-7, wherein the signals delivered by the detector assemblies correspond to one single pulse of neutrons and gamma photons from said source.

25

9. A method for detecting properties of materials, substances or compounds contained in items, comprising the following steps:

- applying a pulsed flux of energetic particles including both neutrons and gamma photons to an item to be analyzed, said particles being
30 intended to react with nuclei of material(s) in said item,
- detecting neutrons and gamma photons coming from said item in response to said application in at least two different energy ranges and in

current detection mode so as to deliver current signals representative of impinging gamma photons and neutrons over time,

- delivering time-resolved current signals representative of neutrons and gamma photons thus detected,

- 5 - generating from time-related features of said signals a signature, and
- comparing said signature with stored reference signatures.

10. The method of claim 9, wherein said neutrons have an energy of at
10 least 3 MeV.

11. The method of claim 9, wherein said neutrons have an energy of at least 5 MeV.

15 12. The method of claim 9, wherein said neutrons have an energy of at least 8 MeV.